

Neutrons and secondary particle analysis in PHITS of HTS components for compact fusion reactors

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dell'Università
e della Ricerca



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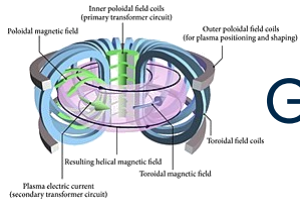
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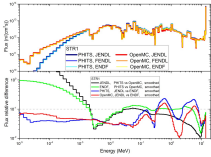
Overview



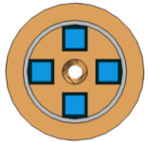
General introduction to the topic



Monte Carlo codes and model description



PHITS–OpenMC cross comparison



VIPER cable analysis



Further developments

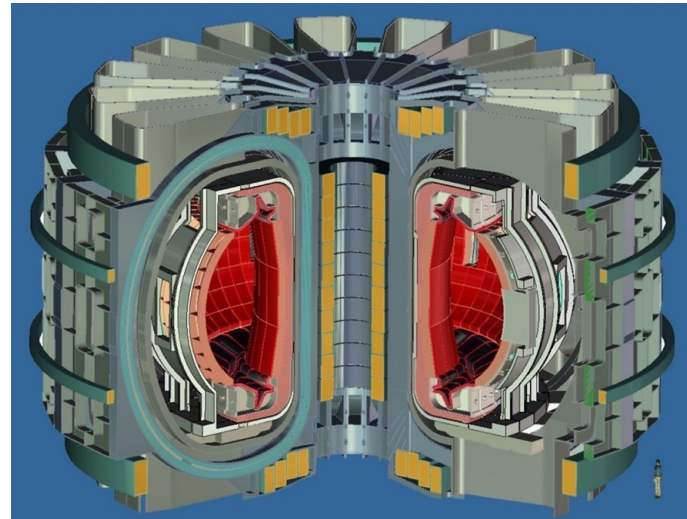
Nuclear fusion: challenges

Extremely high thermal fluxes

Plasma-surfaces interaction

Plasma behaviour and confinement

Radiation environment and damage of materials



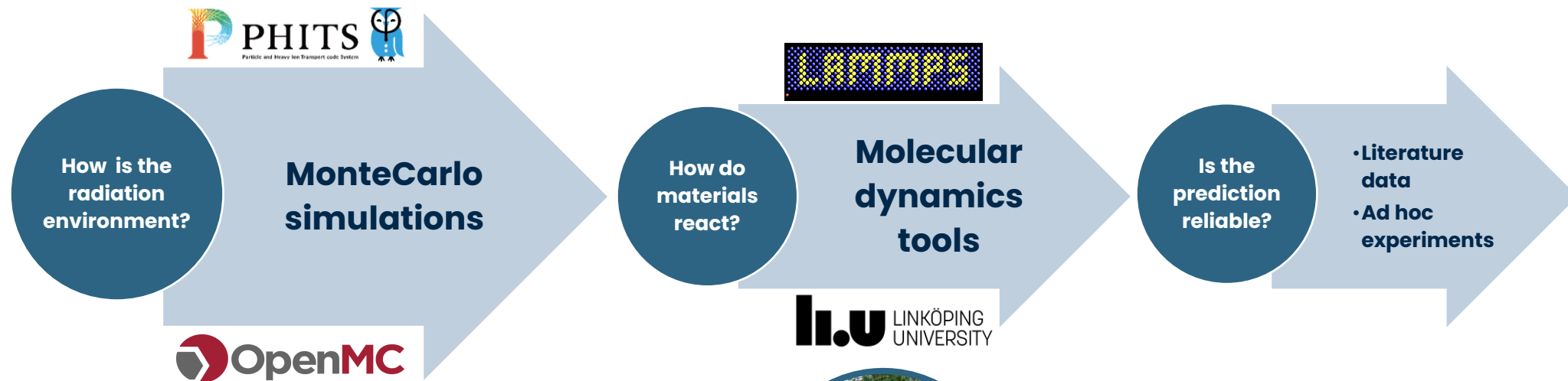
<https://www.iter.org/sci/iterandbeyond>

Power extraction

Tritium breeding and extraction

Radiation environment and damage

- **Main topic** : Modelling the neutron radiation effects on HTS magnets in thermonuclear fusion reactors

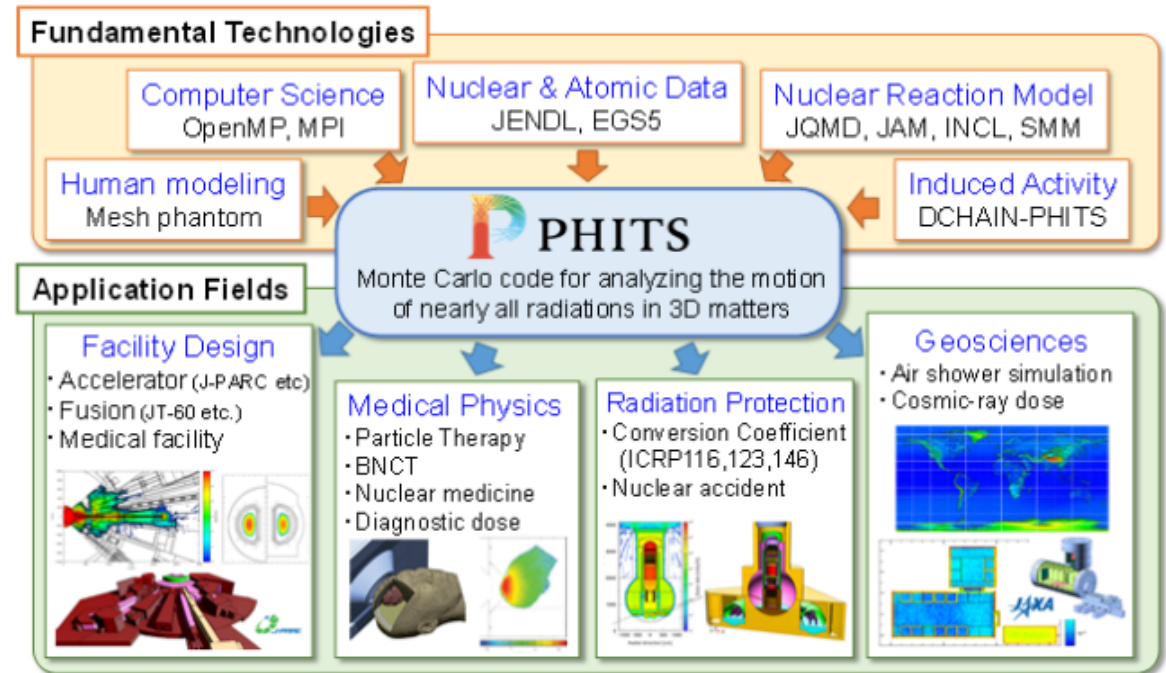


Davide Gambino
Talk today, 17:00 – 17:30

Radiation environment and damage

- To predict the neutron distribution inside the reactor MonteCarlo codes are required

- The PHITS code**, developed by JAEA, was chosen for its **good geometry handling, the possibility of customizing the code, of transporting any particle and of evaluating the dpa directly and compared with the code OpenMC**



source <https://phits.jaea.go.jp/index.html>

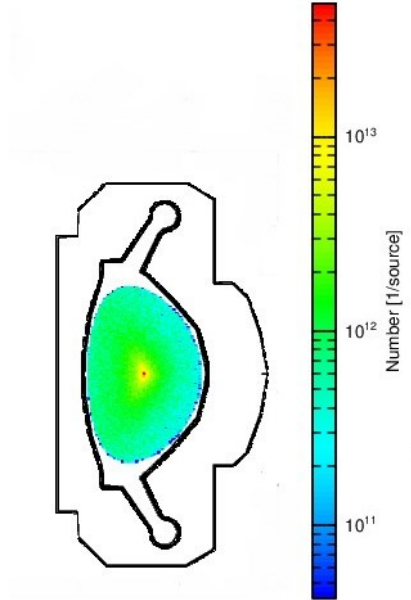
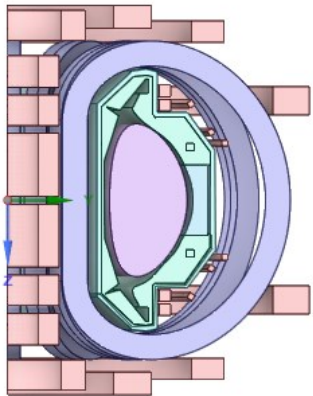


How do neutrons distribute?

Geometry management

Model geometry (CAD conversion)

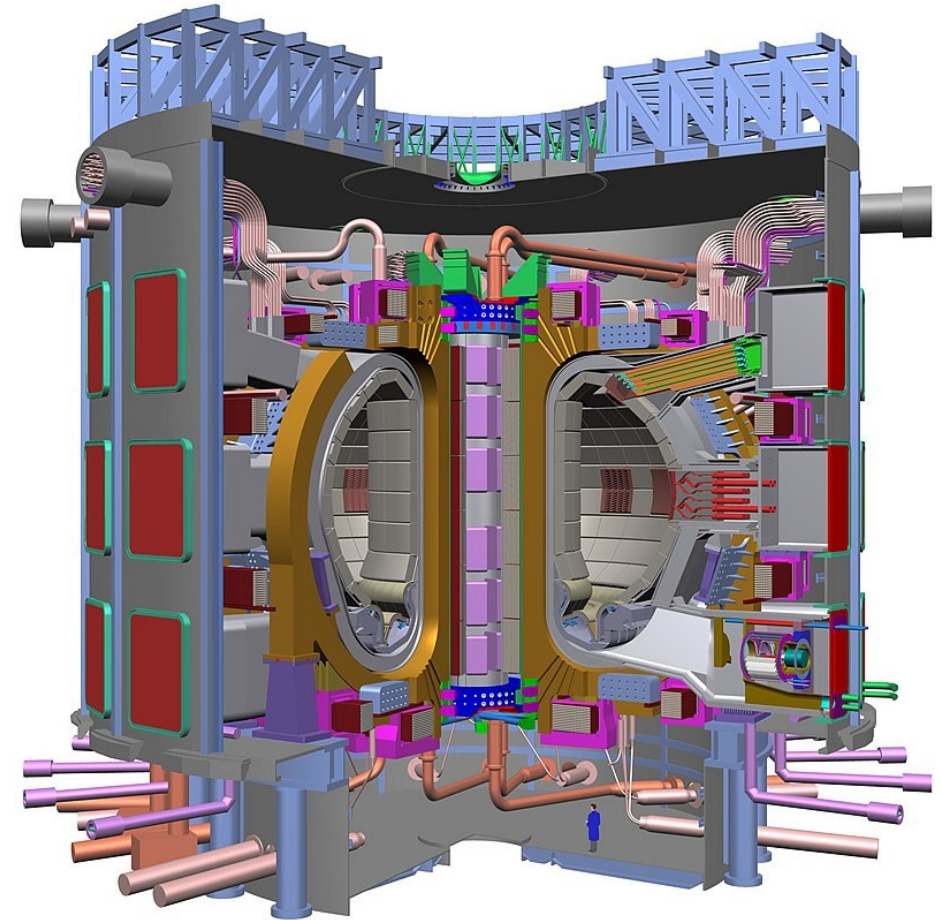
Source geometry declaration (PHITS customization)



Model geometry

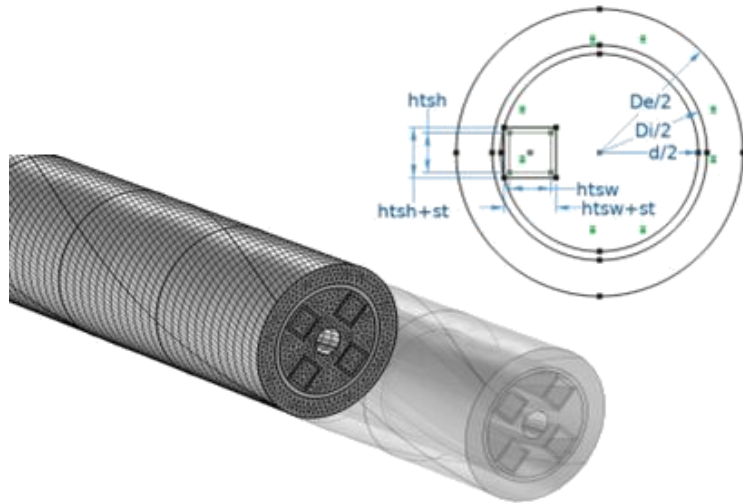
- To perform the transport simulation, the geometry must be declared in PHITS
- The actual geometry of a fusion device is not trivial, declaring it in PHITS by hand using a combination of elementary surfaces is not feasible/ prone to errors and implementation of design changes becomes extremely slow

 **A smarter way is required**



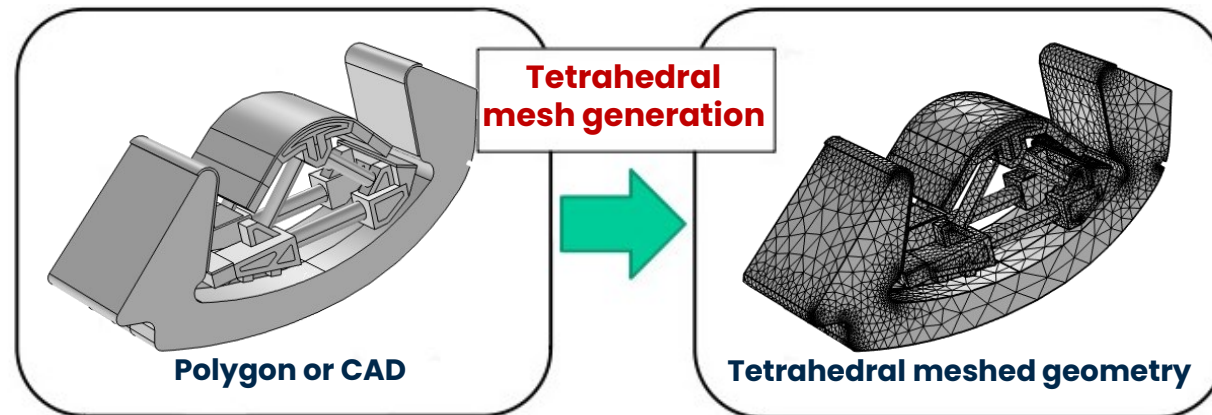
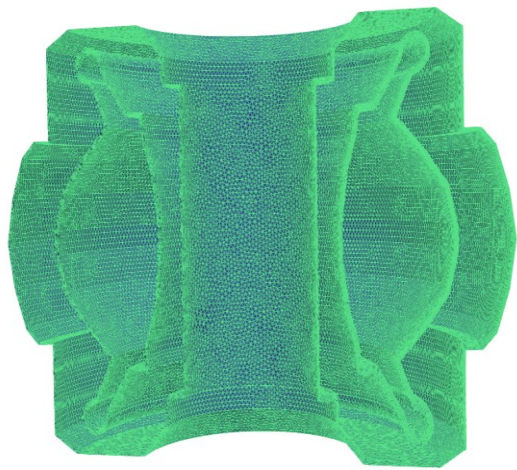
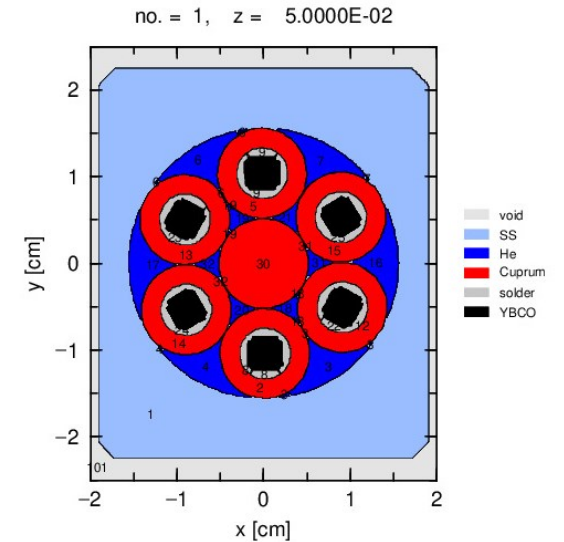
[https://commons.wikimedia.org/wiki/Category:Tokamaks#/media/File:U.S._Department_of_Energy_Science_425_003_001_\(9786811206\).jpg](https://commons.wikimedia.org/wiki/Category:Tokamaks#/media/File:U.S._Department_of_Energy_Science_425_003_001_(9786811206).jpg)

Model geometry

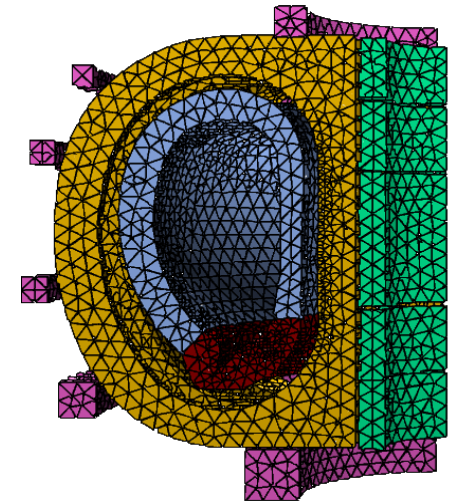


From version 3.30 PHITS supports tetrahedral meshed geometry that can be used to import any CAD.

The DAGMC tool can be used to import CAD in OpenMC.



<https://wias-berlin.de/software/tetgen/>

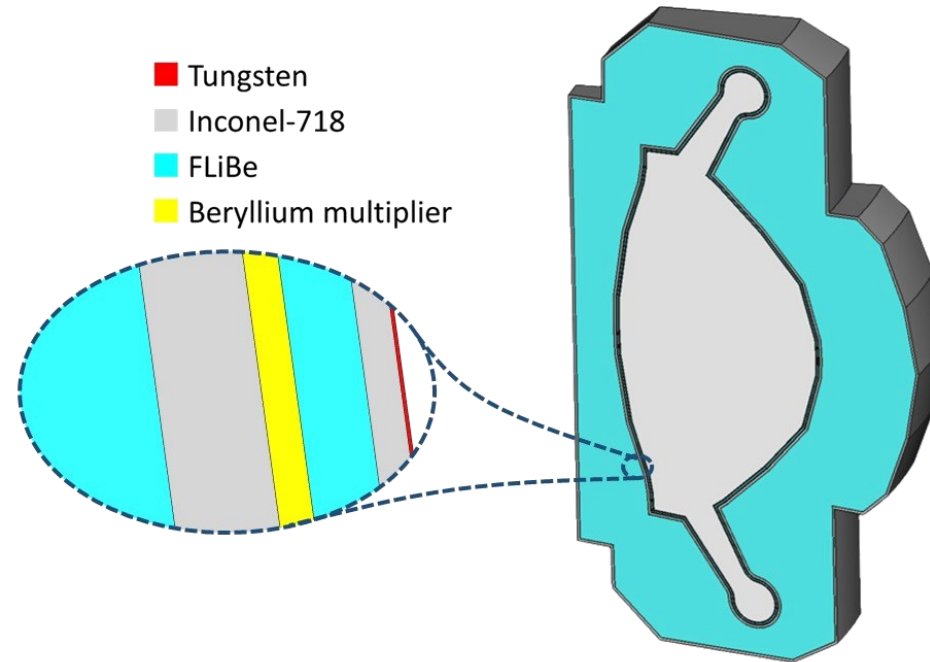


Model geometry

- A good 3D model of reactor Vacuum Vessel(VV) represents the first step of the simulations
- **The PHITS/OpenMC cross comparison was carried out on a 10° degree sector of the reactor with reflective boundary conditions and on a complete 360° model**

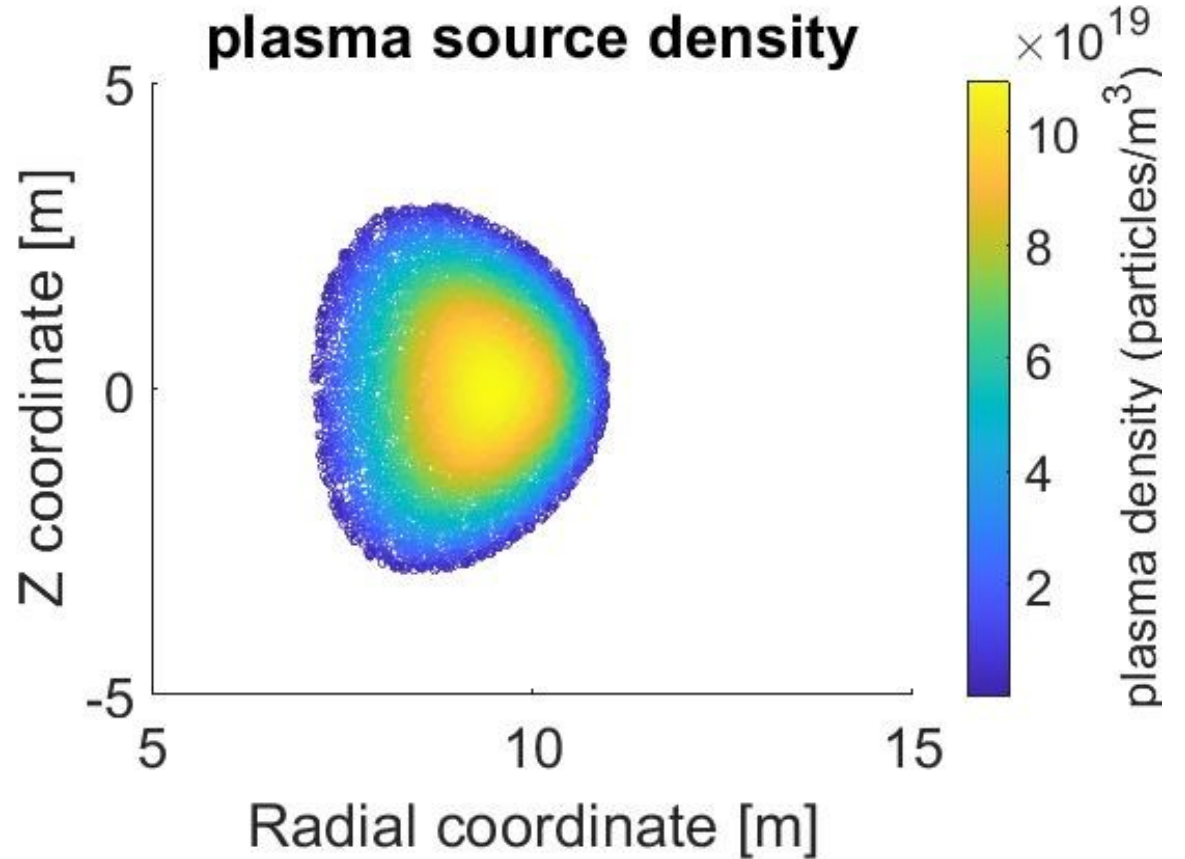
Sorbom et al, Fusion Eng. Des., 2015

Inconel 718						
Element	Ni	Cr	Mo	Nb/Cb	Ti	Al
Mass %	50	17	2.80	4.75	0.65	0.2



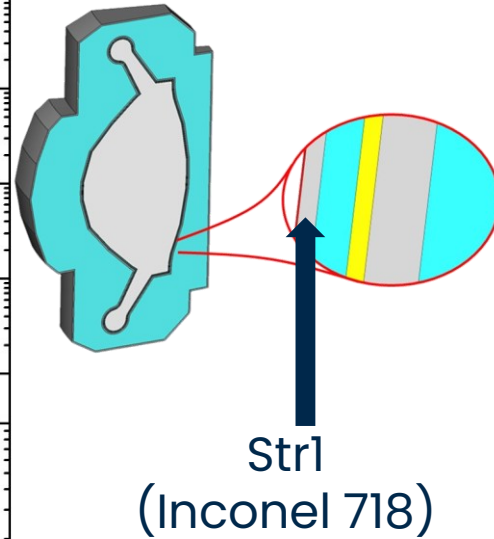
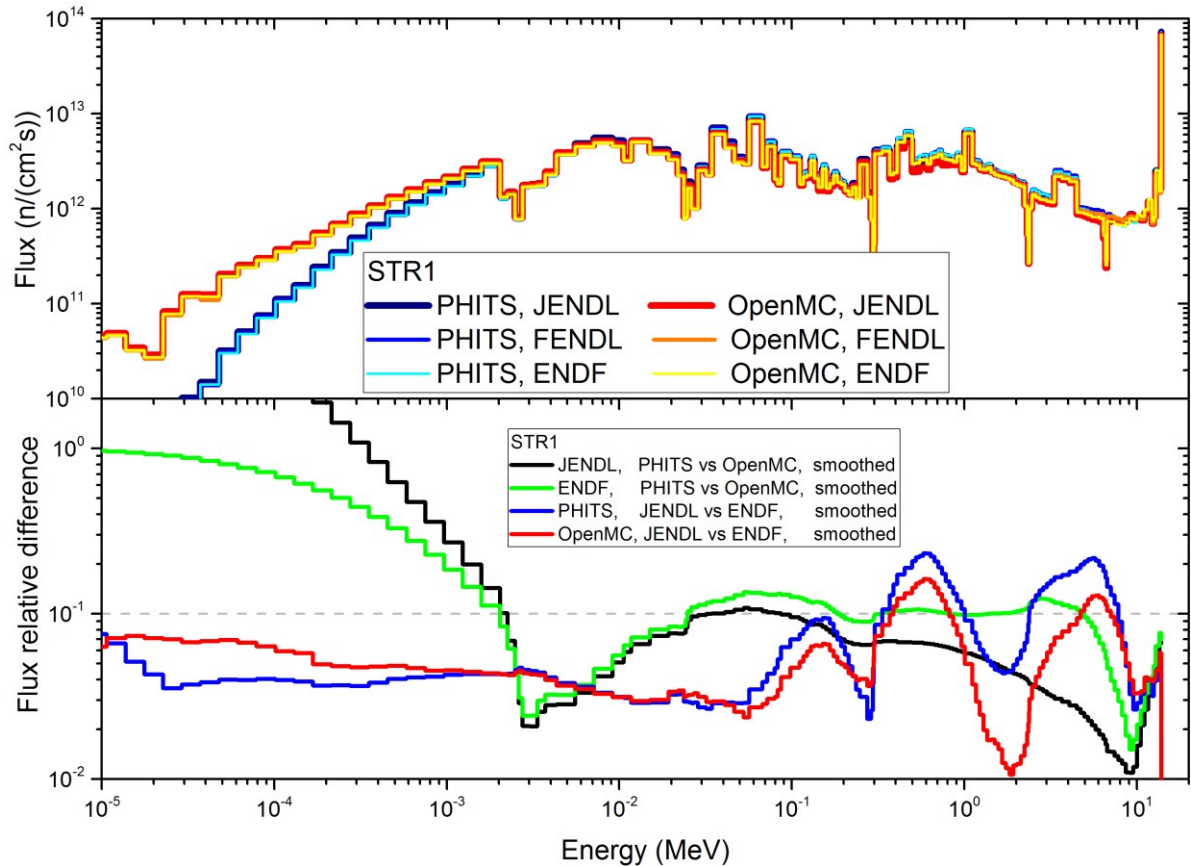
Source geometry

- To perform a particle transport simulation, a particle source is required
- A **toroidal plasma source** was available in OpenMC
- **We introduced a customized plasma source in PHITS**, including all the main physical parameters (e.g Shafranov factor, helicity...)
- Both the sources are based on **Fausser et al., Fusion Eng. Des., 2012** and give the same neutron distribution



Analysis on the VV: spectra

«3D neutronic analysis on compact fusion reactors: PHITS-OpenMC cross-comparison»,
F. Ledda et al., (submitted to *Fusion Engineering and Design*)

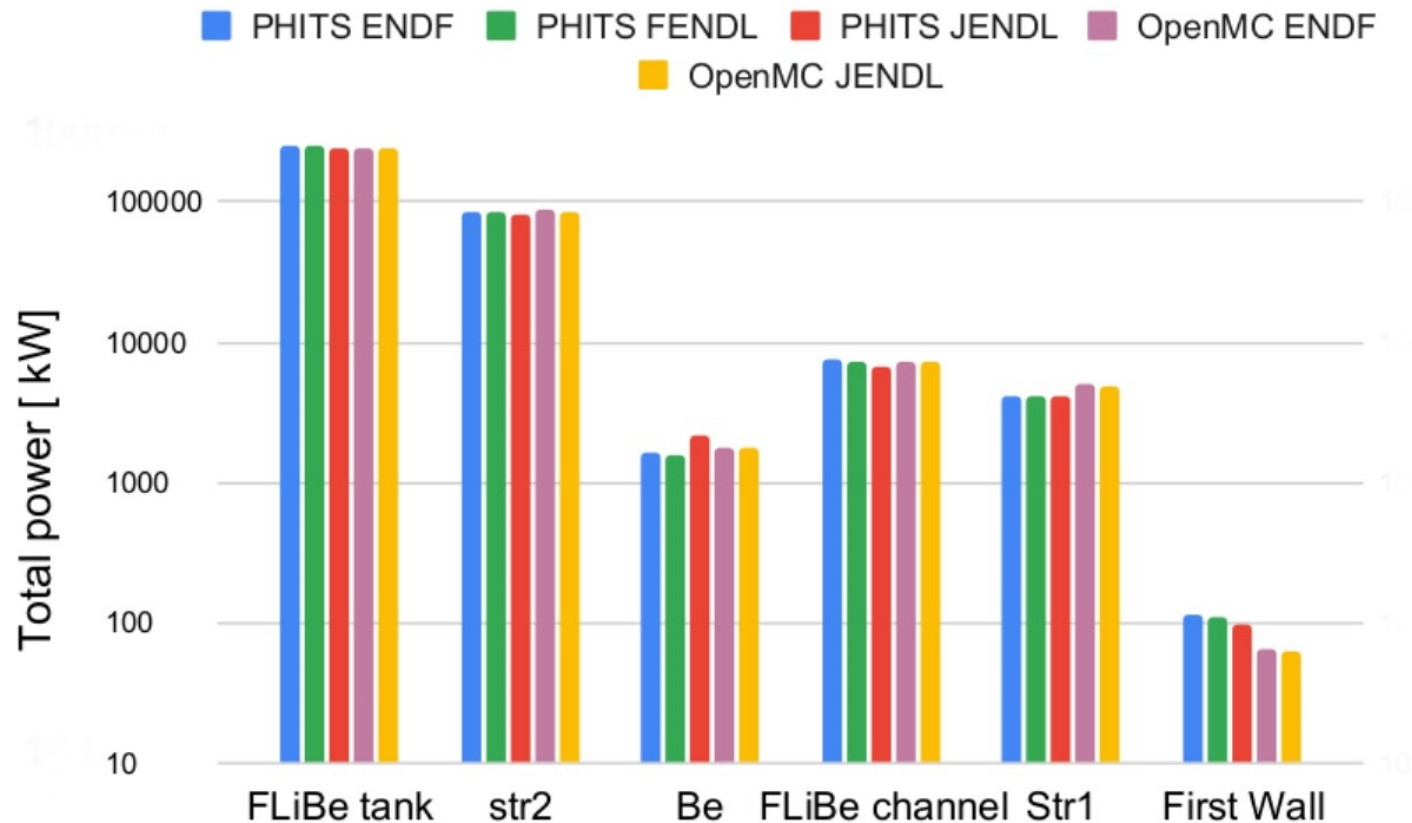


$$\text{Relative difference} = \frac{|\phi_{PHITS} - \phi_{OpenMC}|}{\phi_{reference}}$$

- 3 different nuclear libraries were tested in PHITS and OpenMC, simulating 1 billion neutrons for run
- The use of different codes has the largest impact on the results
- The discrepancy on the low energy region is due to geometry handling issues in DAGMC

Analysis on the VV: power deposition

«3D neutronic analysis on compact fusion reactors: PHITS-OpenMC cross-comparison»,
F. Ledda et al., (submitted to *Fusion Engineering and Design*)



- The total power deposition evaluated with OpenMC and PHITS are in good agreement
- Power deposition on the first wall is affected by the largest discrepancy
- For this case, the best agreement with the average data in literature was obtained with PHITS running with the native JENDL-4.0 library

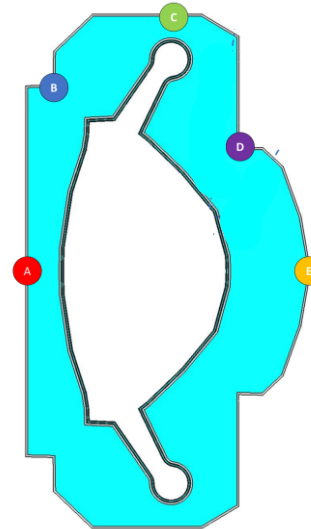
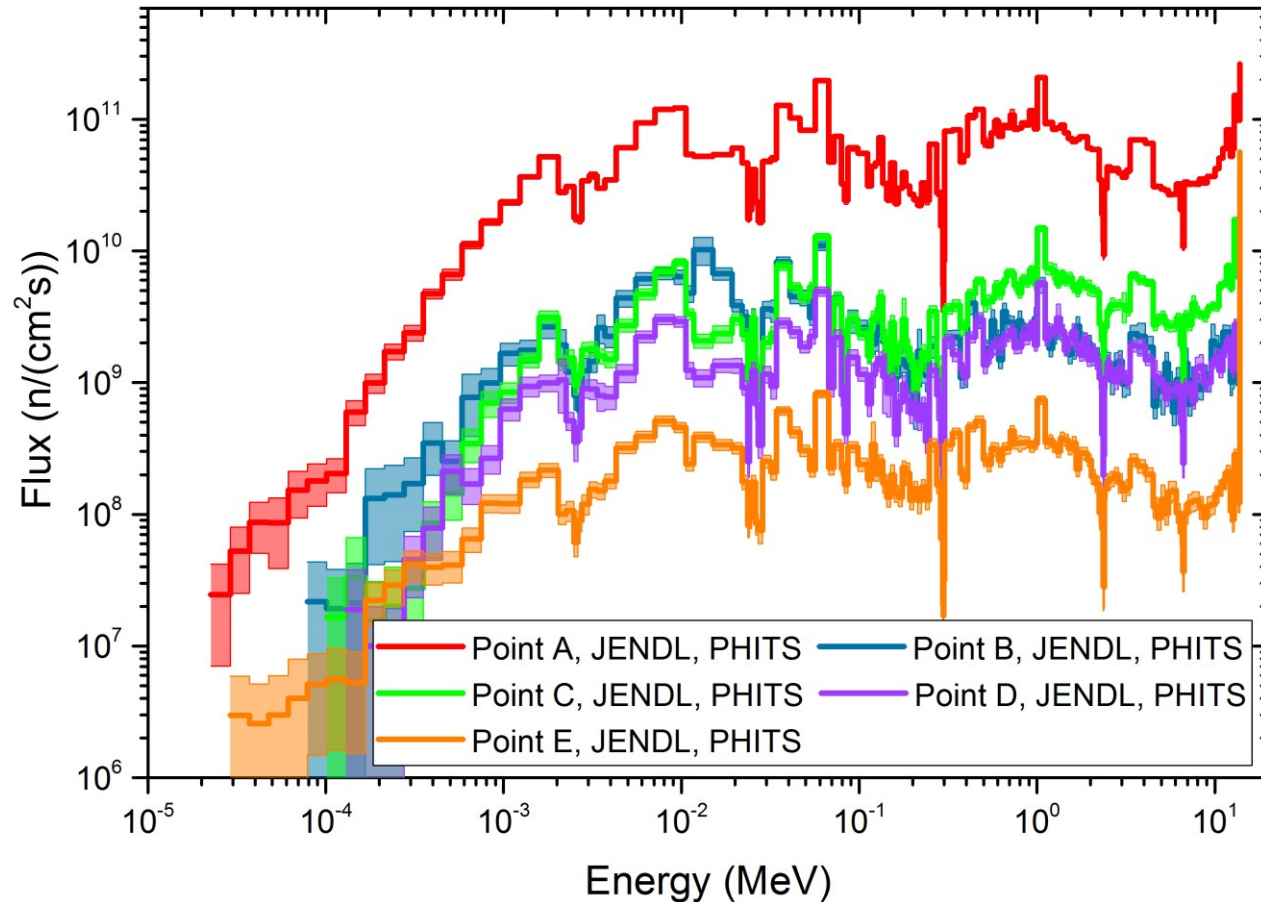
Analysis on the VV: TBR

«3D neutronic analysis on compact fusion reactors: PHITS-OpenMC cross-comparison»,
F. Ledda et al., (submitted to *Fusion Engineering and Design*)

Code	Geometry	Library	TBR value \pm uncertainty
PHITS	Full domain	ENDF/B-VIII.0	1.0766 \pm 0.0001
PHITS	Full domain	JENDL-4.0	1.0736 \pm 0.0001
PHITS	Full domain	FENDL-3.2	1.0703 \pm 0.0001
PHITS	10°domain + reflective BC	ENDF/B-VIII.0	1.0761 \pm 0.0001
OpenMC	Full domain	ENDF/B-VIII.0	1.0626 \pm 0.0001
OpenMC	Full domain	JENDL-4.0	1.0737 \pm 0.0001
OpenMC	Full domain	FENDL-3.2	1.0495 \pm 0.0001
OpenMC	10°domain + reflective BC	ENDF/B-VIII.0	1.0625 \pm 0.0001

- The Tritium Breeding Ratio shows a difference between the two codes of the order of the percent
- Neither the effect of the usage of a different code nor of a different library seems to be prevalent
- The best agreement between the two codes is obtained with the nuclear library JENDL-4.0

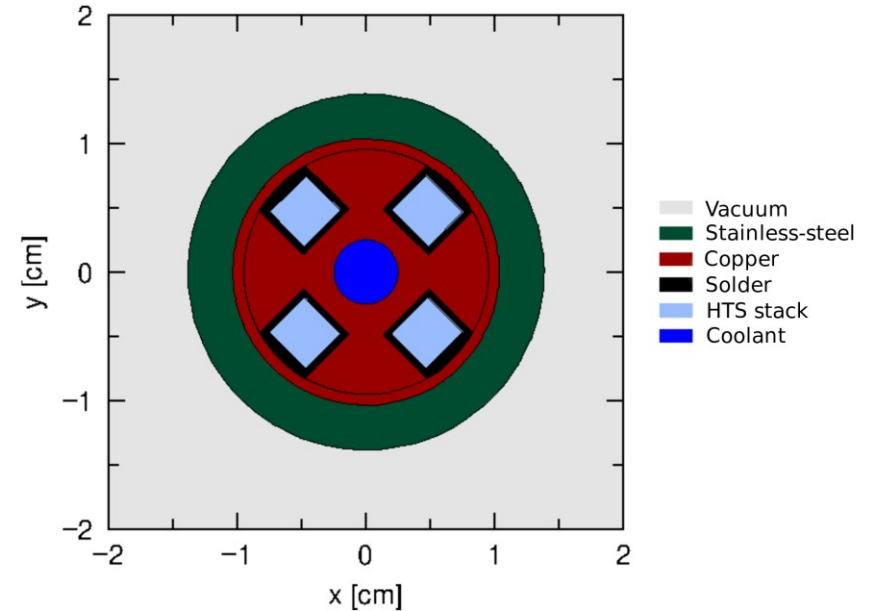
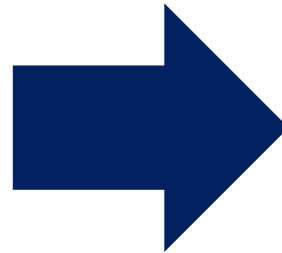
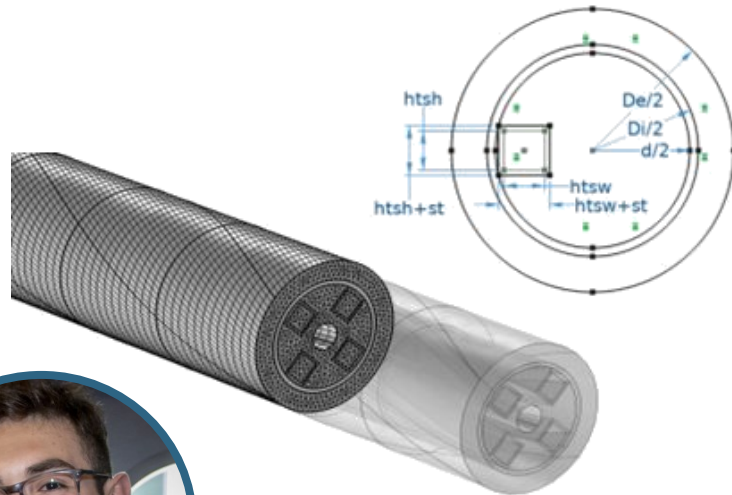
Analysis on the VV



- Neutron spectra were evaluated in 5 poloidal locations at the interface with the TFC position
- Point A is the most critical: this spectrum will be used as input for further analyses of the effects on superconductors

From the VV to the cable

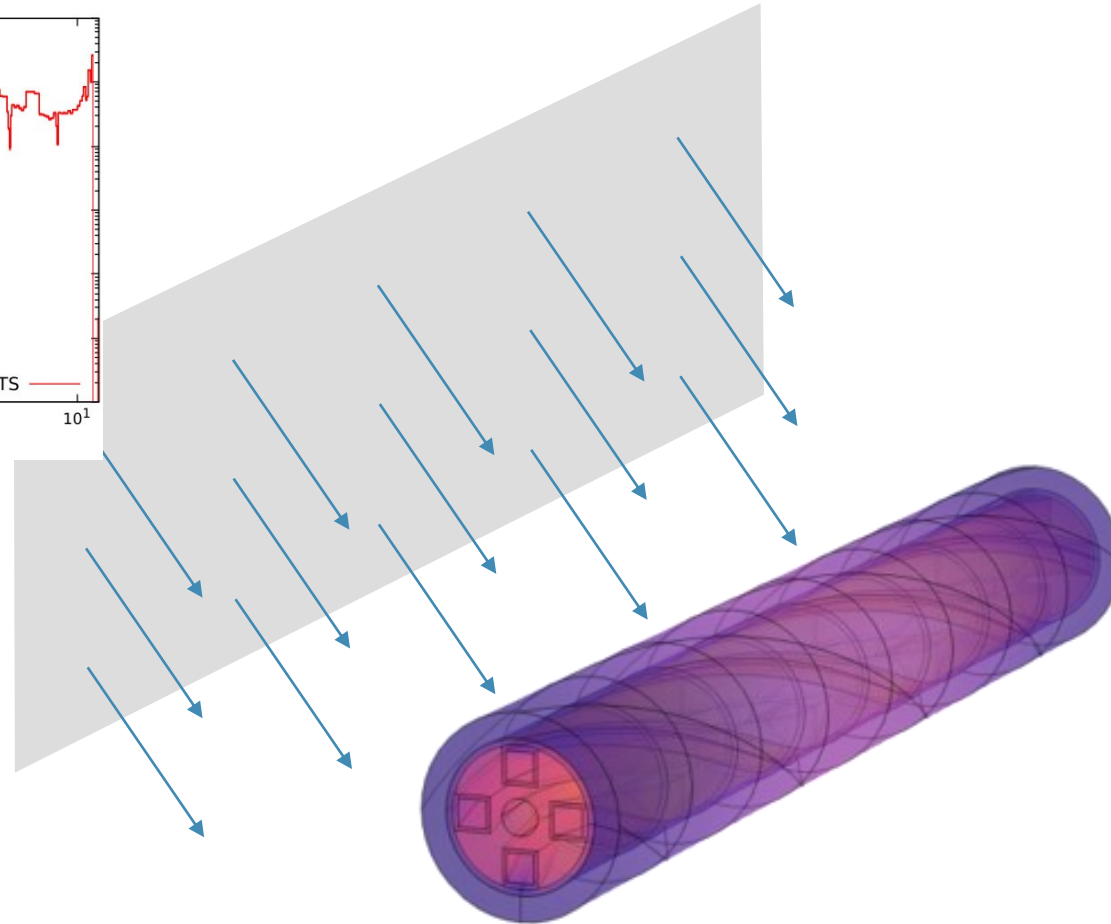
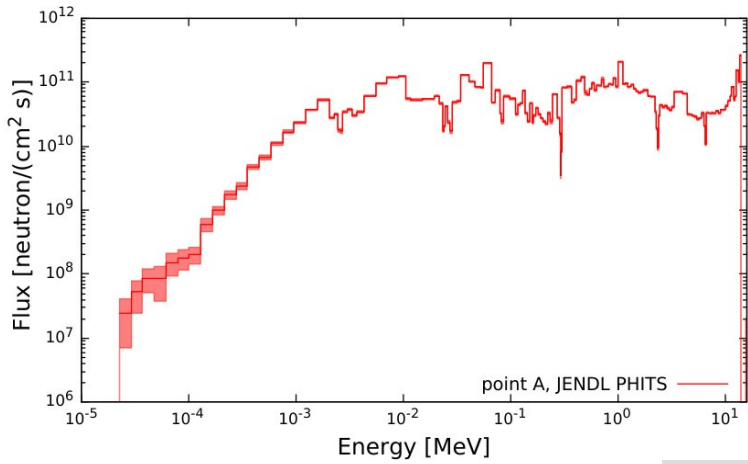
A realistic 3D model of 1 pitch of a superconducting VIPER cable was generated in COMSOL multiphysics® and imported in PHITS



Simone Sparacio

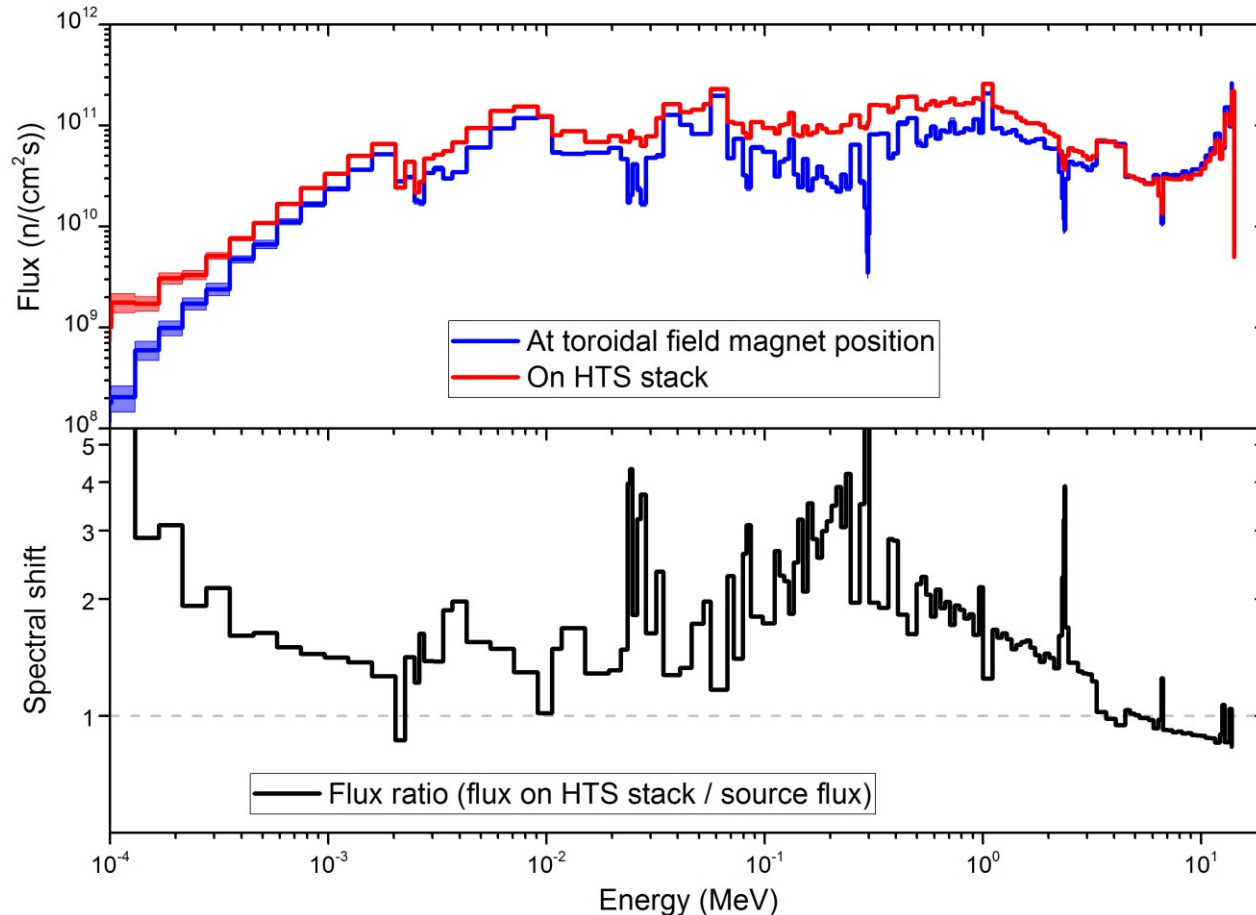
From the VV to the cable

The neutron spectrum evaluated at point A of the reactor geometry was implemented in a planar source emitting collimated neutrons toward the VIPER model



Results on the cable: neutron spectra

“3D neutronic and secondary particles analysis on YBCO tapes for compact fusion reactors”
F. Ledda, D. Torsello et al. (submitted to IEEE Transactions on Applied Superconductivity)



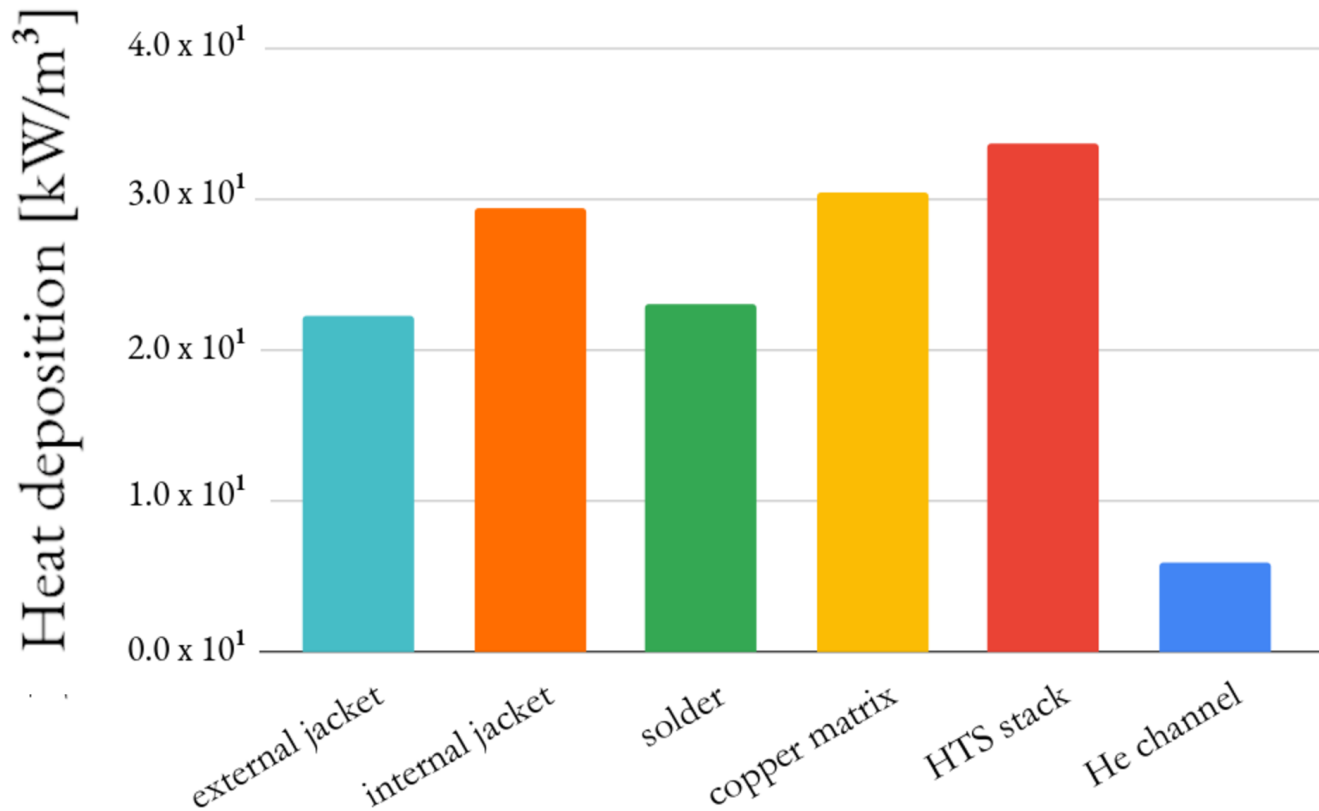
- We observe an enhancement of the neutron flux on the HTS stacks and a spectral shift
- Material choice should be optimized for the nuclear environment

Presented at Eucas 2023 (poster)

Results on the cable: power deposition

“3D neutronic and secondary particles analysis on YBCO tapes for compact fusion reactors”

F. Ledda, D. Torsello et al. (submitted to **IEEE Transactions on Applied Superconductivity**)



- For nuclear load evaluation, HTS composition details were deduced from Superpower[®] tapes
- The HTS stack is the most loaded element in the VIPER cable from a power deposition point of view.
- Further thermal analysis in **S.Sparacio et al., submitted to IEEE Trans. Appl. Supercond., talk tomorrow, 12:30 – 13:00**

Results on the cable: dpa

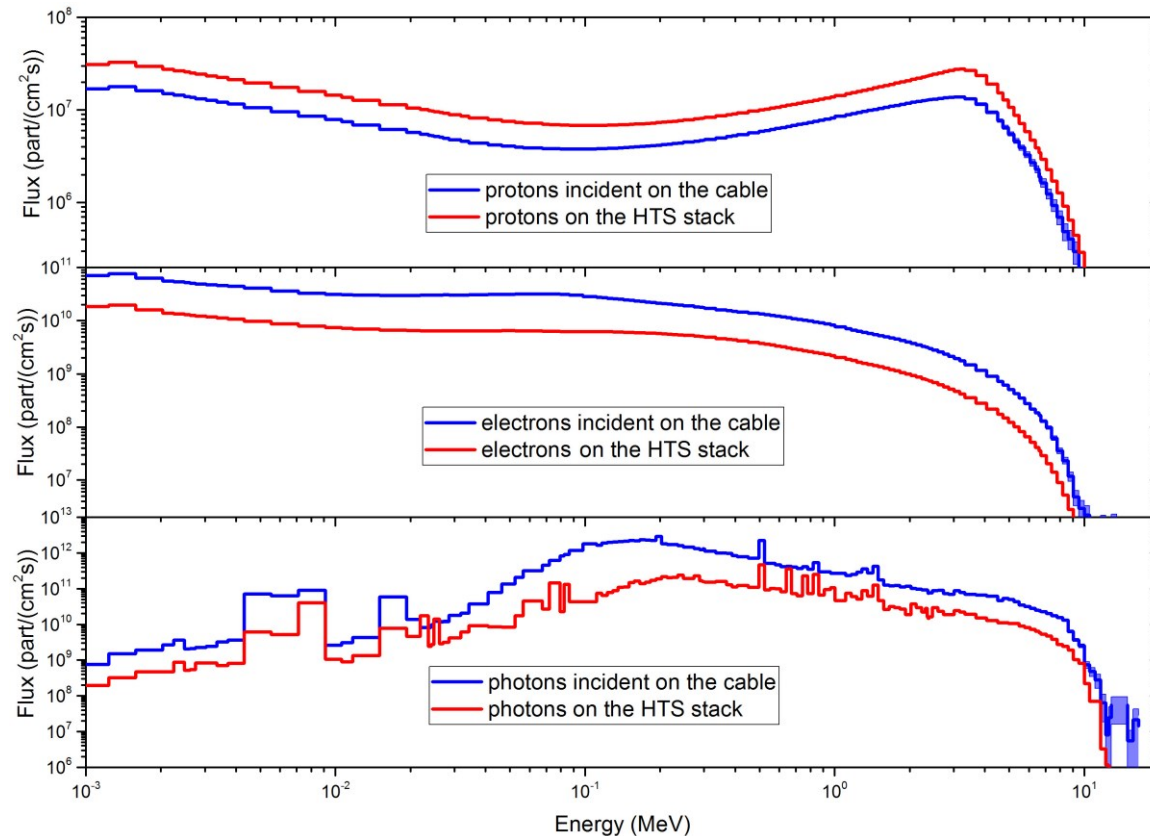
“3D neutronic and secondary particles analysis on YBCO tapes for compact fusion reactors”
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Quantity	Value
dpa after 10 years	0.21 JENDL 4.0
	0.23 ENDF-VIII/B
	0.23 FENDL 3.2
Neutron heat deposition (kW/m ³)	33.6
Decay heat after 10 years (kW/m ³)	0.82

- The HTS stack was assumed to be composed of pure YBCO for dpa evaluation
- Dpa after 10 years supports the order of magnitude and refines a previous estimate (**D. Torsello et al., 2023 Supercond. Sci. Technol., 36 014003**)
- Decay heat is negligible when compared with direct neutron load

Results on the cable: secondary particles

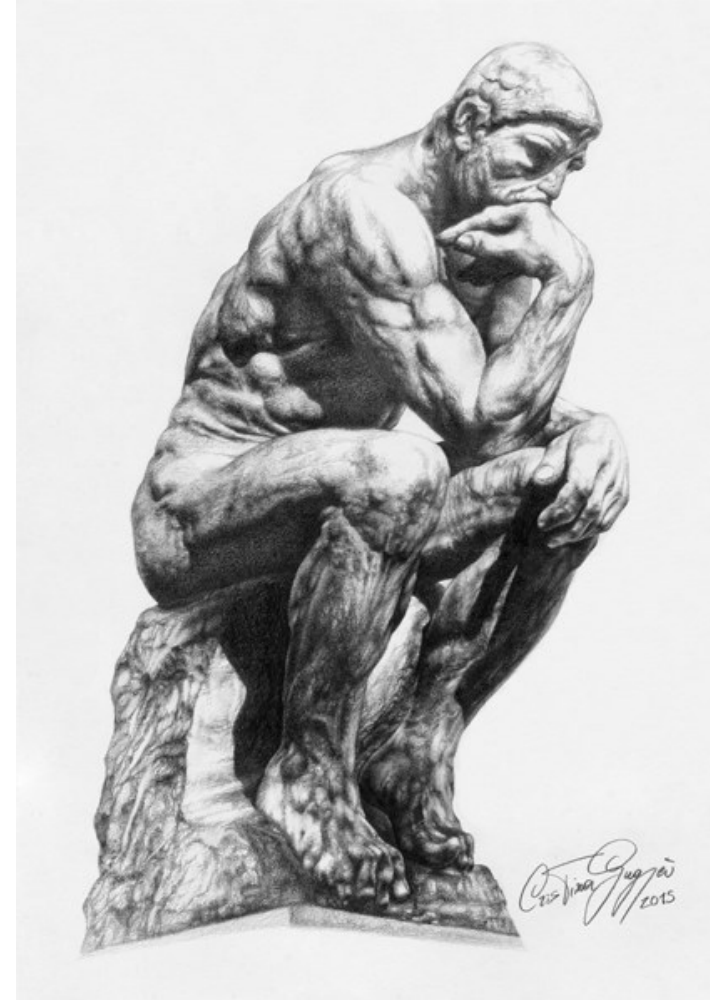
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F. Ledda, D. Torsello et al. (submitted to IEEE Transactions on Applied Superconductivity)



- Secondary particles (among which photons, electrons and protons) are generated in the cable, with not negligible fluxes
- Their effect on HTS performance during reactor operations should be considered: **experiments of superconductivity performance under irradiation are crucial**

What next ?

- Exploit the MC capability to consider complex geometries and sources, analyzing different cable designs
- Refine the reactor model, including penetrations, ports and instrumentations
- Introduction of more sophisticated dpa formulations, considering also athermal recombination (e.g. ARC-dpa)
- PKA spectra evaluation directly in the MC code
- Employ the MC simulation for reactor design optimization



Thanks for your attention!



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